



U.S. Department of Transportation

Pipeline and Hazardous Materials Safety Administration

# COMPETENT AUTHORITY CERTIFICATION FOR A TYPE AF

FISSILE RADIOACTIVE MATERIALS PACKAGE DESIGN CERTIFICATE USA/9239/AF, REVISION 16

This certifies that the radioactive materials package design described below has been certified by the Competent Authority of the United States as meeting the regulatory requirements for a Type AF packaging for fissile radioactive materials as prescribed in the regulations of the International Atomic Energy Agency and the United States of America<sup>2</sup>.

- 1. Package Identification Model Nos. MCC-3, MCC-4 and MCC-5.
- Package Description and Authorized Radioactive Contents as described in U.S. Nuclear Regulatory Commission Certificate of Compliance No. 9239, Revision 15 (attached).

### 3. Criticality -

- a. Minimum Criticality Safety Index: 0.4
- b. The maximum number of packages shall be determined in accordance with Table X of the IAEA's "Regulations for the Safe Transport of Radioactive Materials, 1996 Edition (Revised), No. TS-R-1 (ST-1 Revised).

### 4. General Conditions -

- a. Each user of this certificate must have in his possession a copy of this certificate and all documents necessary to properly prepare the package for transportation.
- b. Each user of this certificate, other than the original petitioner, shall register his identity in writing to the Radioactive Materials Branch (DHM-23), Office of Hazardous Materials Technology, Pipeline and Hazardous Materials Safety Administration, U.S. Department of Transportation, Washington, D.C. 20590-0001.
- c. This certificate does not relieve any consignor or carrier from compliance with any requirement of the Government of any country through or into which the package is to be transported.
- 5. Marking and Labeling The package shall bear the marking USA/9239/AF in addition to other required markings and labeling.

<sup>&</sup>quot;Regulations for the Safe Transport of Radioactive Materials, 1996
Edition (Revised)", No. TS-R-1 (ST-1, Revised)," published by the
International Atomic Energy Agency (IAEA), Vienna, Austria

 $<sup>^{2}</sup>$  Title 49, Code of Federal Regulations, Parts 100 - 199, United States of America.

## CERTIFICATE USA/9239/AF, REVISION 16

6. Expiration Date - This certificate expires on March 31, 2012.

This certificate is issued in accordance with paragraph 814 and 816 of the IAEA Regulations and Sections 173.471 and 173.472 of Title 49 of the Code of Federal Regulations, in response to the February 12, 2007 petition by the Westinghouse, Columbia, SC and in consideration of other information on file in this Office.

Certained by

MAR 23 2007

(DATE)

Bob Richard

Deputy Associate Administrator for Hazardous Materials Safety

Revision 16 - Issued to endorse U.S. Nuclear Regulatory Commission Certificate of Compliance No. 9239, Revision 15.

NRC FORM 618 (8-2000) 10 CFR 71		TE OF COMPL		ULATORY	/ СОММІ	ISSION
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#### 2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.
- 3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION
  - a. ISSUED TO (Name and Address)
     Westinghouse Electric Company LLC (WELCO)
     P.O. Box 355
     Pittsburgh, PA 15230
- b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION
  Westinghouse Electric Corporation application
  dated August 29, 2006, as supplemented.

### 4. CONDITIONS

5.

This certificate is conditional upon fallining the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

(a) Packaging

(1) Model Nos.: MCC 3 MCC-4, and MCC-5

(2) Description

The MCC packages are shipping containers for unirradiated uranium oxide fuel assemblies. The packagings container for unirradiated uranium oxide fuel assembly equipped with a strongback and an adjustable fuel element clamping assembly. The cradle assembly is shock mounted to a 13-gauge carbon steel outer container by shear mounts. The MCC-3 container is closed with thirty ½-inch T-bolts. The MCC-4 and MCC-5 containers are closed with fifty ½-inch T-bolts.

The MCC-3 and MCC-4 containers are permanently equipped with vertical  $\mathrm{Gd_20_3}$  neutron absorber plates that are mounted on the center wall of the strongback. Additional horizontal  $\mathrm{Gd_20_3}$  neutron absorber plates, mounted on the underside of the strongback, are required for the contents as specified.

The MCC-5 container is permanently equipped with both the vertical and horizontal  $Gd_2O_3$  neutron absorber plates. Additional vee-shaped, guided  $Gd_2O_3$  neutron absorber plates are required for the contents as specified.

Approximate dimensions of the MCC-3 packaging are 44½ inches O.D. by 194½ inches long. The gross weight of the packaging and contents is 7,544 pounds. The maximum weight of the contents is 3,300 pounds.

Approximate dimensions of the MCC-4 packaging are 44½ inches O.D. by 226 inches long. The gross weight of the packaging and contents is 10,533 pounds. The maximum weight of the contents is 3,870 pounds.

NRC FORM 618 (8-2000) 10 CFR 71  CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES  U.S. NUCLEAR REGULATORY COMMISSION  CERTIFICATE OF COMPLIANCE								
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## 5. (a) (2) Packaging (continued)

Approximate dimensions of the MCC-5 packaging are 44½ inches O.D. by 226 inches long. The gross weight of the packaging and contents is 10,533 pounds. The maximum weight of the contents is 3,700 pounds.

## (3) Drawings

The MCC-3 packaging is constructed in accordance with Westinghouse Electric Corporation Drawing No. MCCL301, Sheets 1, 2, 3, and 4, Rev. 6.

The MCC-4 packaging is constructed in accordance with Westinghouse Electric Corporation Drawing No. MCCL401, Sheets 1, 2, 3, 4, and 5, Rev. 9.

The MCC packaging is constructed in accordance with Westinghouse Electric Corporation Drawing No. MCCL501, Sheets 1 through 10, Rev. 6.

## (b) Contents

(1) Type and form of material

Unimadiated PWR uranium dioxide fuel assemblies with a maximum uranium-235 entichment of 5.0 weight percent with the following exceptions: 15x15 BW fuel assemblies pave a maximum entitlinent of 4.65 wt%, and VVER-1000 fuel assemblies have a maximum entitlinent of 4.65 wt%.

The fuel assemblies shall meeting specifications given in Westinghouse Drawing No. 6481 15, Rev. 4, and in the tollowing tables of Appendix 1-5 of the application, as supplemented:

Table 1-5.1, Rev. 12*	Fuel Assembly Parameters 4x14 Type Fuel Assemblies	
Table 1-5.2, Rev. 12*	Fuel Assembly Parameters 15x15 Type Fuel Assemblies	
Table 1-5.3, Rev. 12*	Fuel Assembly Parameters 16x16 Type Fuel Assemblies**	
Table 1-5.4, Rev. 12*	Fuel Assembly Parameters 17x17 Type Fuel Assemblies**	
Table 1-5.5, Rev. 12*	Fuel Assembly Parameters VVER-1000 Type Fuel Assembly***	

As submitted by letter dated January 24, 2007.

<sup>16</sup>x16 CE fuel assemblies and the 17x17 W-STD/XL fuel assemblies shall be shipped only in the Model No. MCC-4 package.

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5. (b) (1) Contents (continued)

\*\*\* VVER-1000 fuel assemblies shall be shipped only in the Model No. MCC-5 package.

(2) Maximum quantity of material per package

Two (2) fuel assemblies

(c) Transport Index for Criticality Control (Criticality Safety Index)

Minimum transport index to be shown on label for nuclear criticality control:

0.4

- 6. (a) For shipments of 4x14, 15x15, 16x16, and 17x17 OFA fuel assemblies with U-235 enrichments of over 4.65 wt% and up to 5.0 wt%, horizontal Go. neutron absorber plates shall be positioned underneath each assembly. The horizontal absorber plates shall be placed horizontally on the underside of the strongback, as specified in the respective drawings in Condition 5(a) for the MCC-3 and MCC-4 models.
  - (b) For shipments of 17x17 STANDARD lattice fuel assemblies (17x17 STD and 17x17 XL) with U-235 enrichments of over 4.85 wt% and up to 5.0 wt%, horizontal Gd<sub>2</sub>0<sub>3</sub> neutron absorber plates shall be positioned underneath each assembly. The horizontal absorber plates shall be placed horizontally on the underside of the strongback, as specified in the respective drawings in Condition 3(a)(3) for the MCC-3 and MCC-4 models.
- 7. Shipments of VVER-1000 fuel assemblies are authorized with U-235 eprichments up to 4.80 wt%.
- 8. Each fuel assembly must be unsheathed or must be enclosed in an unsealed plastic sheath which may not extend beyond the ends of the fuel assembly. The ends of the sheath may not be folded or taped in any manner that would prevent flow of liquids into or out of the sheathed fuel assembly.
- 9. The dimensions, minimum  $Gd_2O_3$  loading and coating specifications, and acceptance testing of the neutron absorber plates shall be in accordance with the " $Gd_2O_3$  Neutron Absorber Plates Specifications," Appendix 1-7, Rev. 12, of the application, as supplemented. The minimum  $Gd_2O_3$  coating areal density on the vertical and horizontal neutron absorber plates shall be  $0.054 \text{ g-}Gd_2O_3/\text{cm}^2$ . The minimum  $Gd_2O_3$  coating areal density on guided neutron absorber plates shall be  $0.027 \text{ g-}Gd_2O_3/\text{cm}^2$ .
- 10. In addition to the requirements of Subpart G of 10 CFR Part 71:
  - (a) Each package shall be prepared for shipment and operated in accordance with the "Routine Shipping Container Utilization Summary Operating Procedures," in Chapter 7 of the application, as supplemented; and
  - (b) Each package shall be tested and maintained in accordance with the "Acceptance Tests, Maintenance Program, and Recertification Program," in Chapter 8 of the application, as supplemented, and as specified in the respective drawings in Condition 5(a)(3) for the MCC-3, MCC-4, and MCC-5 models.

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- The package authorized by this certificate is hereby approved for use under the general license 11. provisions of 10 CFR §71.17.
- Revisions No. 14 of this certificate may be used until January 31, 2008. 12.
- 13. Expiration date: March 31, 2012.

## **REFERENCES**

Westinghouse Electric Corporation application dated August 29, 2006.

Supplement dated: September 25, November 9, 2006, and January 24, 2007.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Meraj Rahimi, Acting Chief

Licensing Branch

Division of Spenishiel Storage and Transportation Office of Tyuclear Waterial/Safety

Date: February 2, 2007





Pipeline and Hazardous Materials Safety Administration

CERTIFICATE NUMBER: USA/9239/AF-85, Revision 16

## **ORIGINAL REGISTRANT(S):**

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### **REGISTERED USER(S):**

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